

## Integrated modelling of the main Divertor Tokamak Test facility scenarios

I. Casiraghi<sup>1,2</sup>, P. Mantica<sup>2</sup>, F. Koechl<sup>3</sup>, R. Ambrosino<sup>4,5,6</sup>, L. Aucone<sup>7</sup>, B. Baiocchi<sup>2</sup>,  
L. Balbinot<sup>8,9</sup>, A. Castaldo<sup>5</sup>, J. Citrin<sup>10,11</sup>, M. Dicorato<sup>1</sup>, L. Frassinetti<sup>12</sup>, A. Mariani<sup>1</sup>,  
P. Vincenzi<sup>8</sup>, P. Agostinetti<sup>8</sup>, S. Ceccuzzi<sup>6,13</sup>, L. Figini<sup>2</sup>, G. Granucci<sup>2</sup>, P. Innocente<sup>8</sup>,  
T. Johnson<sup>12</sup>, H. Nyström<sup>12</sup>, M. Valisa<sup>8</sup>

<sup>1</sup>Università degli Studi di Milano-Bicocca, Milano, Italy; <sup>2</sup>Istituto per la Scienza e Tecnologia dei Plasmi, CNR, Milano, Italy; <sup>3</sup>CCFE, Culham Science Centre, Abingdon, UK; <sup>4</sup>Università degli Studi di Napoli Federico II, Napoli, Italy; <sup>5</sup>Consorzio CREATE, Napoli, Italy; <sup>6</sup>DTT S.C. a r.l., Frascati, Italy; <sup>7</sup>Politecnico di Milano, Milano, Italy; <sup>8</sup>Consorzio RFX, Padova, Italy; <sup>9</sup>Università degli Studi di Padova, Padova, Italy; <sup>10</sup>DIFFER - Dutch Institute for Fundamental Energy Research, Eindhoven, NL; <sup>11</sup>Science and Technology of Nuclear Fusion Group, Eindhoven University of Technology, Eindhoven, NL; <sup>12</sup>Fusion Plasma Physics, ECSS, KTH Royal Institute of Technology, Stockholm, Sweden; <sup>13</sup>ENEA C.R.Frascati, Frascati, Italy.

### Introduction

The Divertor Tokamak Test facility (DTT) [1, 2] is a new D-shaped superconducting tokamak under construction in Italy, with the first plasma planned for 2026. It will be equipped with 3 auxiliary heating systems: a 170 GHz ECRH system, a 60 – 90 MHz ICRH system, and a negative ion NBI system. The primary task of DTT ( $R_0 = 2.19\text{m}$ ,  $a = 0.70\text{m}$ , pulse length  $\leq 100\text{s}$ ,  $B_T \leq 6\text{T}$ ,  $I_{\text{pl}} \leq 5.5\text{MA}$ ,  $P_{\text{sep}}/R \simeq 15$ ) is to study the controlled power and particle exhaust from a fusion reactor, which is a main research topic in the European Fusion Roadmap [3], and test alternative exhaust strategies.

An intensive integrated modelling work of DTT operational scenarios with the Single Null (SN) divertor configuration (Fig.1) is underway, in order to support the machine design, and particularly the definition of the heating mix, the design of the neutron shields, the assessment of fast particle losses and the design of diagnostic systems, as well as to help the elaboration of a DTT scientific work-programme.

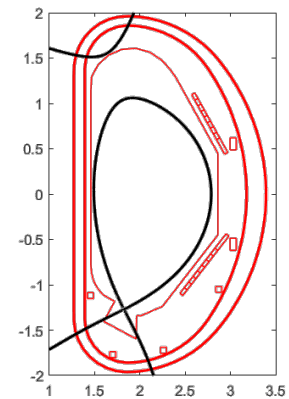


Figure 1: Plasma shape of the SN DTT scenario.

### Integrated modelling set-up

Simulations of the main operational scenarios in H-mode were performed using first-principle based transport models and state-of-art modules for heating, fuelling and magnetic equilibrium, and are described in detail in [4]. So far, the Full Power (FP), Day-1, and Day-0 scenarios have

been simulated, using the JINTRAC[5] suite of codes or the ASTRA[6] transport solver with a mixed ASTRA–JINTRAC approach. These simulations predict steady-state radial profiles of electron and ion temperature, density, current density, rotation, power depositions, and impurities (Ar and W) in  $\rho_{\text{tor}} \leq 0.94$ .

The pedestal was calculated by EPED1[7]. The turbulent heat and particle transport is calculated by the Trapped-Gyro-Landau-Fluid (TGLF)[8] or QuaLiKiz (QLK)[9] quasi-linear transport models. Particularly, TGLF SAT1-geo (released in Nov.2019) and TGLF SAT2 (released in Jan.2021)[10] have been compared to the new QLK release 2.8.1[11] and an “ad hoc” QLK version (specifically developed for DTT to match gyrokinetic predictions in TEM dominant conditions). The results of the modelling performed with the previous device design with  $R_0 = 2.14\text{m}$  and  $a = 0.65\text{m}$  will be presented and discussed.

### Full Power scenario

The optimisation of the power distribution in the FP scenario amongst the auxiliary heating systems and of the NBI energy was one of the purposes of this modelling work. By the comparison of several alternatives, the reference heating mix of the FP scenario has been selected:  $\sim 28.8\text{MW}$  of ECRH coupled power,  $\sim 6\text{MW}$  of ICRH coupled power, and  $\sim 10\text{MW}$  of NBI coupled power with a single beam at  $510\text{keV}$ . In the top of Fig. 2 the radial profiles of  $T_e$ ,  $T_i$ ,  $n_e$ ,  $\omega_{\text{tor}}$ , and  $q$  obtained for this reference FP scenario using various transport models are compared. We note that, according to all models, DTT operates in a regime with  $T_e/T_i > 1$ . The 4 models agree reasonably outside  $\rho_{\text{tor}} = 0.4$ , whilst QLK features much flatter density inside  $\rho_{\text{tor}} = 0.4$ , where ECH power density is very high. By a comparison of QLK and TGLF stand-alone runs with gyrokinetic simulations done with GENE, in the inner region the flat QLK  $n_e$  peaking is not validated and the TGLF  $n_e$  peaking appears a bit overestimated, but qualitatively nearer to GENE predictions.

To have a reference  $q_{95} > 3$ , the device has been enlarged to  $R_0 = 2.19\text{m}$  and  $a = 0.70\text{m}$ . The updating to the

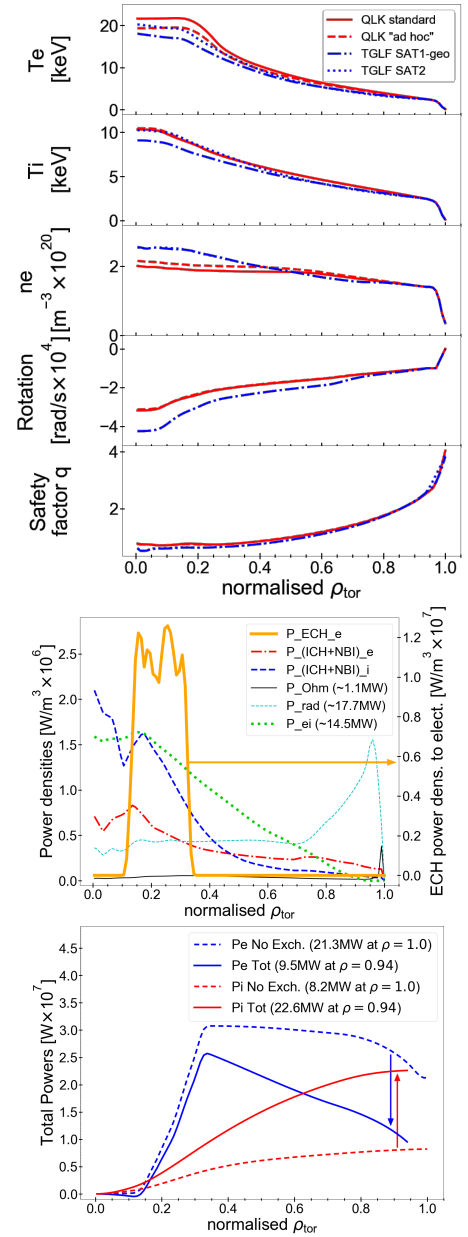


Figure 2: FP reference scenario profiles.

new DTT size of the modelling work is underway. Globally, the Ohmic power  $P_{\text{Ohm}} \approx 1.1\text{MW}$  is quite negligible, the core radiated power  $P_{\text{rad}} \approx 15\text{--}18\text{MW}$  is about the 35% of the total power, and a large amount of power ( $P_{\text{rad}} \approx 14\text{--}15\text{MW}$ ) is exchanged from electrons to ions because of the collisional coupling. Some impurity penetrations into the core is observed with both models. In the FP reference scenario, the energy fraction owned by the energetic particles due to both NBI and ICRH systems amounts to  $W_{\text{EP}}/W_{\text{tot}} \approx 6.5\text{--}7.7\%$ .

The neutron rate resulted  $\leq 1.5 \times 10^{17}/\text{s}$ , i.e. compatible with the present design of neutron shields.

## Fuelling issues

To investigate the edge neutral level required to operate in the FP scenario without pellets, the standard QLK run has been extended up to the separatrix, including an edge transport barrier tuned to reproduce the previous pedestal. Since the neutral density rate, shown in Fig.3, is significant up to  $\rho_{\text{tor}} \sim 0.8$ , the neutral penetration into the plasma evaluated by FRANTIC[12] is adequate for fuelling. The NBI contribution to the neutral source is small. To reach the required density value at the top of barrier, we need at the separatrix a neutral flux level of  $\sim 0.36 \times 10^{22}$  particles/s, which corresponds to a D fuelling of  $\sim 5 \times 10^{22}/\text{s}$ . Being near to the feasibility limit, a pellet injection system is deemed useful as a fuelling method in DTT to avoid degrading the edge plasma with extremely high gas puff rates and to minimise the operational risk.

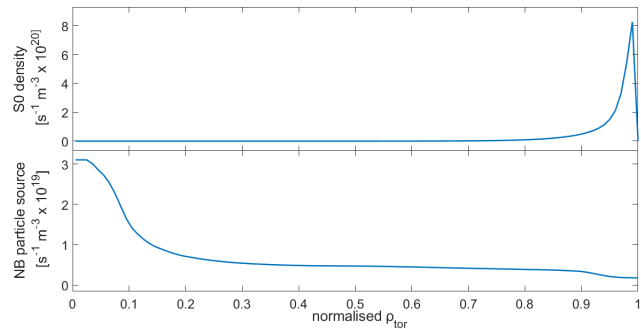


Figure 3: Profiles of neutral sources in the FP case.

## Day-1 scenario

In the Day-1 scenario (with  $B_{\text{tor}} = 6\text{T}$  and  $I_{\text{pl}} = 4.0\text{MA}$ ), about 25MW of external power is coupled with the plasma: 14.4MW from ECRH in first harmonic O-mode, 3MW from ICRH, and 7.5MW at plasma from NNBI with 1 beam at 400keV. According to both TGLF and QLK,  $T_i$  values are similar in

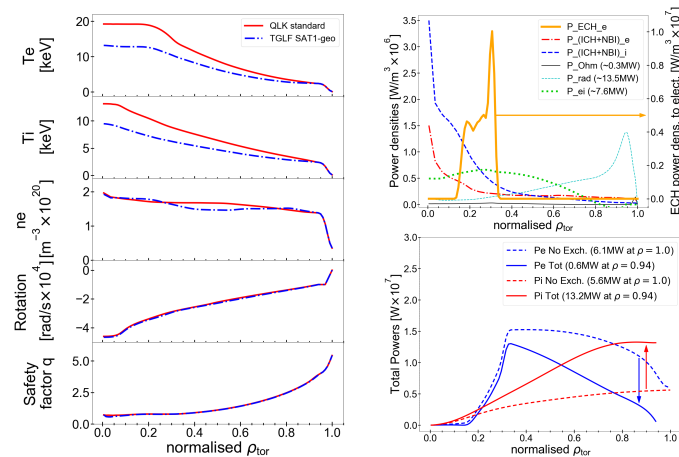


Figure 4: Radial profiles for the Day-1 phase.

Day-1 and FP scenarios, due to the high ion stiffness, while the  $n_e$  peaking is less pronounced in Day-1 than in the FP case. Some discrepancies in the  $T_e$  and  $T_i$  profiles appear between the

two models. We notice that  $T_e$  is significantly reduced in the Day-1 TGLF run with respect to the FP profile.

### Day-0 scenario

The day-0 phase (with  $B_{\text{tor}} = 3\text{T}$  and  $I_{\text{pl}} = 2.0\text{MA}$ ), i.e. the first experimental plasma, features only 8 MW of ECH power in second harmonic X-mode to maximise the absorption. Since the day-0 scenario is a purely electron heated case, this is the scenario where QLK validity is most affected by the dominant TEM regime. Therefore, modelling results displayed in Fig.5 refer only to the TGLF SAT1-geo simulation.

Due to having only electron heating and low density,  $T_e$  is much larger than  $T_i$  ( $T_{e0} \approx 12\text{ keV}$  and  $T_{i0} \approx 4\text{ keV}$ ).

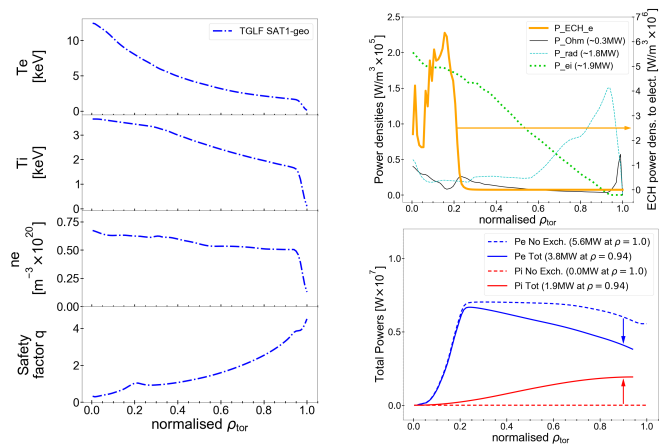


Figure 5: *Radial profiles for the Day-0 phase.*

### Conclusions

Integrated steady-state simulations of the main DTT scenarios are now available. The machine size has been increased, the heating mix has been defined, neutron shields have been confirmed, and a pellet injection system is deemed useful as a fuelling method in DTT.

### References

- [1] DTT Interim Design Report, ENEA (2019), <https://www.dtt-dms.enea.it/share/s/avvghVQT2aSkSgV9vuEtw>
- [2] Special Section of Fusion Engineering and Design, Vol. 122, 253-294, e1-e25 (2017)
- [3] A.J.H. Donné, Phil. Trans. R. Soc. A 377 20170432 (2018) Special Section of Fusion Engineering and Design, Vol. 122, 253-294, e1-e25 (2017)
- [4] I. Casiraghi, P. Mantica, F. Koechl et al., First-principle based multi-channel integrated modelling in support to the design of the Divertor Tokamak Test facility, submitted to Nuclear Fusion
- [5] M. Romanelli et al., Plasma and Fusion Research, 9, 3403023 (2014)
- [6] G.V. Pereverzev et al., ASTRA automated system for transport analysis in a tokamak IPP Report 5/98
- [7] P.B. Snyder et al., Nuclear Fusion, 51(10):103016 (2011)
- [8] G. M. Staebler et al., Physics of Plasmas, 23(6):062518 (2016)
- [9] J. Citrin, C. Bourdelle, and F. J. Casson et al., Plasma Physics and Controlled Fusion, 59(12):124005 (2017)
- [10] G. M. Staebler et al., Plasma Phys. Control. Fusion 63 015013 (2020)
- [11] C. Stephens et al. to be submitted to Phys. Plasmas
- [12] S. Tamor., J. Comput. Phys., 40(104) (1981)